

## **Appendix A**

### **Waste Profile**



## WASTE PROFILE

WASTE PROFILE

PART I

A. GENERAL INFORMATION

WASTE PROFILE NO.

1. GENERATOR NAME

2. FACILITY ADDRESS/LOCATION

3. 20 X 10 X 10 TEMP

4. WAG ID & Uniform Waste Stream

5. TECHNICAL CONTACT

6. TITLE

7. PHONE

8. e-mail

B. 1. NAME OF WASTE

2. USEPA/or/STATE WASTE CODE(S)

3. PROCESS GENERATING WASTE

4. PROJECTED ANNUAL VOLUME/UNITS

5. MODE OF COLLECTION

6. IS THIS WASTE A DODGE LISTED WASTE AS DEFINED IN 40 CFR 261.31?

7. IS THIS WASTE RESTRICTED FROM LAND DISPOSAL BY 40 CFR 268?

8. HAS AN EXEMPTION BEEN GRANTED?

9. DOES THE WASTE MEET APPLICABLE TREATMENT STANDARDS?

PART II

1. MATERIAL CHARACTERIZATION

COLOR (required)

DENSITY

POTENTIAL SOLIDS

ASH CONTENT

MULTILAYERED

BILAYERED

2. RCRA CHARACTERISTICS

PHYSICAL STATE:

TREATMENT GROUP:

FLASH POINT (F)

WATER REACTIVE

CYANIDE REACTIVE

4. MATERIAL COMPOSITION

COMPONENT

CONCENTRATION

RANGE

___ LOW TOC (< 10%) ___ CORROSIVE (D002) pH _____ ___ CORRODES STEEL	___ SULFIDE REACTIVE ___ TOXICITY CHARACTERISTIC (SEE PART III)	TOTAL _____ 100% <b>5. SHIPPING INFORMATION</b> DOT HAZARDOUS MATERIAL? YES ___ NO ___ PROPER SHIPPING NAME _____ HAZARD CLASS _____ U.N. OR N.A. NO. _____ ADDITIONAL DESCRIPTION _____ METHOD OF SHIPMENT: ___ BULK ___ DRUM CERCLA REPORTABLE QUANTITY (RQ) _____ EMERGENCY RESPONSE GUIDE PAGE _____ DOT PUBLICATION 39HA _____ PAGE NO. _____ SPECIAL HANDLING INFORMATION _____
<b>3. CHEMICAL COMPOSITION (ppm or mg/L)</b>		
COPPER _____ NICKEL _____ ZINC _____ CHROMIUM-HEX _____ (OTHER) _____	PHENOLICS _____ TOTAL HALOGENS _____ VOLATILE ORGANICS _____ PCBs _____	
NOTE: EXPLOSIVES, SHOCK-SENSITIVE, PYROPHORIC, AND ETIOLOGICAL WASTE NORMALLY MAY NOT BE ACCEPTED BY THE SSA DESIGNER WITHOUT SPECIAL APPROVAL.		
<b>6. GENERATOR INFORMATION</b>		
BASIS FOR INFORMATION ___ ANALYTICAL ANALYSIS (ATTACH RESULTS) ___ OTHER (ACKNOWLEDGE WITH ATTACHED SUPPORTING DOCUMENTS; explain how and why these documents comply with RCRA requirements.)		
I HEREBY CERTIFY THAT ALL INFORMATION SUBMITTED IN AND ALL ATTACHED DOCUMENTS IS TO THE BEST OF MY KNOWLEDGE AN ACCURATE REPRESENTATION OF THE WASTE TURNED IN TO THE SSA. ALL KNOWN OR SUSPECTED HAZARDS HAVE BEEN DISCLOSED.		
SIGNATURE OF GENERATOR'S REPRESENTATIVE _____ <small>(Print of Title Name)</small>		DATE _____
<b>7. WASTE ACCEPTANCE INTO ICDF landfill    SSTF    <input type="checkbox"/> Evaporation Pond</b>		
SIGNATURE OF ICDF Complex DESIGNEE _____ <small>Final Acceptance</small>		DATE _____
SIGNATURE OF ICDF Complex DESIGNEE _____ <small>Final Acceptance</small>		DATE _____

PART III					
HAZARDOUS CHARACTERISTIC LIST					
<input type="checkbox"/> Total Metals <input type="checkbox"/> TCLP* <input type="checkbox"/> Process Knowledge					
CONTAMINANT	EPA HW No.	(mg/L)	CONTAMINANT	EPA HW No.	(mg/L)
— ARSENIC	D004		— HEXACHLORO-1,3-BUTADIENE	D033	
— BARIUM	D005		— HEXACHLOROETHANE	D034	
— BENZENE	D018		— LEAD	D008	
— CADMIUM	D006		— MERCURY	D013	
— CARBON TETRACHLORIDE	D019		— METHOXYCHLOR	D009	
— CHLORDANE	D020		— METHYL ETHYL KETONE	D014	
— CHLOROBENZENE	D021		— NITROBENZENE	D035	
— CHLOROFORM	D022		— PENTACHLOROPHENOL	D036	
— CHROMIUM	D007		— PYRIDINE	D038	
— O-CRESOL	D023		— SELENIUM	D010	
— M-CRESOL	D024		— SILVER	D011	
— P-CRESOL	D025		— TETRACHLOROETHYLENE	D039	
— 2,4-D	D026		— TOXOPHENE	D015	
— 1,2-DICHLOROBENZENE	D027		— TRICHLOROETHYLENE	D040	
— 1,2-DICHLOROETHANE	D028		— 2,4,5-TRICHLOROPHENOL	D041	
— 1,1-DICHLOROETHYLENE	D029		— 2,4,6-TRICHLOROPHENOL	D042	
— 2,4-DINITROTOLENE	D030		— 2,45-TP (SILVEX)	D017	
— ENDRIN	D031		— VINYL CHLORIDE	D043	
— HEPTACHLOR AND ITS HYDROLYSIS	D032				
— HEXACHLOROBENZENE					

\*TCLP data are required for waste streams where total metals exceed 20X the TCLP LDRs.

All required analysis for this sheet must be attached prior to submittal.

## RADIOLOGICAL LIST

ISOTOPE	%	(pCi/g)	ISOTOPE	%	(pCi/g)
<sup>3</sup> H			<sup>60</sup> Co		
<sup>7</sup> Be			<sup>60</sup> Co act. metal <sup>c</sup>		
<sup>10</sup> Be			<sup>63</sup> Ni		
<sup>14</sup> C			<sup>63</sup> Ni act. metal <sup>c</sup>		
<sup>14</sup> C act. Metal <sup>c</sup>			<sup>65</sup> Zn		
<sup>22</sup> Na			<sup>68</sup> Ge		
<sup>32</sup> P			<sup>75</sup> Se		
<sup>35</sup> S			<sup>76</sup> Se		
<sup>36</sup> Cl			<sup>86</sup> Sr		
<sup>40</sup> K			<sup>86</sup> Kr		
<sup>45</sup> Ca			<sup>86</sup> Sr		
<sup>46</sup> Sc			<sup>87</sup> Rb		
<sup>48</sup> V			<sup>89</sup> Y		
<sup>51</sup> Cr			<sup>90</sup> Sr		
<sup>54</sup> Mn			<sup>90</sup> Mo		
<sup>55</sup> Fe			<sup>90m</sup> Nb		
<sup>56</sup> Co			<sup>93</sup> Zr		
<sup>57</sup> Co			<sup>94</sup> Nb		
<sup>58</sup> Co			<sup>94</sup> Nb act. <sup>c</sup>		
<sup>58</sup> Fe			<sup>95</sup> Nb		
<sup>59</sup> Ni					
<sup>59</sup> Ni act. Metal <sup>c</sup>					
<sup>99</sup> Tc			<sup>207</sup> Bi		
<sup>100</sup> Ru			<sup>210</sup> Pb		
<sup>100</sup> Ru			<sup>210</sup> Po		
<sup>107</sup> Pd			<sup>226</sup> Ra		
<sup>100m</sup> Ag			<sup>227</sup> Ac		
<sup>109</sup> Cd			<sup>228</sup> Ra		
<sup>110m</sup> Ag			<sup>228</sup> Th		
<sup>113</sup> Sn			<sup>229</sup> Th		
<sup>114</sup> Sn			<sup>230</sup> Th		
<sup>121</sup> Sn			<sup>231</sup> Pa		
<sup>121</sup> Te			<sup>232</sup> Th		
<sup>123</sup> Te			Total U		
<sup>124</sup> Sb			<sup>232</sup> U		
<sup>125</sup> I			<sup>233</sup> U		
<sup>126</sup> Sb			<sup>234</sup> Th		
<sup>126</sup> Sb			<sup>234</sup> U		
<sup>127</sup> Te			<sup>235</sup> U		
<sup>127</sup> Te			<sup>236</sup> Pu		
<sup>129</sup> I			<sup>236</sup> U		
<sup>129</sup> Te			<sup>237</sup> Np <sup>d</sup>		
<sup>131m</sup> Xe			<sup>238</sup> Pu <sup>d</sup>		
			<sup>238</sup> U		
			<sup>239</sup> Pu <sup>d</sup>		

RADIOLOGICAL LIST (continued)					
ISOTOPE	%	(pCi/g)	ISOTOPE		(pCi/g)
— <sup>133</sup> Ba			— <sup>240</sup> Pu		
— <sup>134</sup> Cs			— <sup>241</sup> Pu		
— <sup>135</sup> Cs			— <sup>241</sup> Am		
— <sup>137</sup> Cs— <sup>137m</sup> Ba			— <sup>242</sup> Am		
— <sup>140</sup> Ba			— <sup>242</sup> Cm		
— <sup>141</sup> Ce			— <sup>243</sup> Am		
— <sup>144</sup> Ce— <sup>144</sup> Pr			— <sup>243</sup> Cm		
— <sup>147</sup> Nd			— <sup>244</sup> Cm		
— <sup>147</sup> Pm			— <sup>244</sup> Pu		
— <sup>147</sup> Sm			— <sup>245</sup> Cm		
— <sup>150</sup> Eu			— <sup>246</sup> Cm		
— <sup>151</sup> Sm			— <sup>247</sup> Am		
— <sup>152</sup> Eu			— <sup>247</sup> Cm		
— <sup>152</sup> Gd			— <sup>248</sup> Cm		
— <sup>153</sup> Gd			— <sup>249</sup> Cf		
— <sup>154</sup> Eu			— <sup>250</sup> Cf		
— <sup>155</sup> Eu			— <sup>250</sup> Cm		
— <sup>170</sup> Tm			— <sup>251</sup> Cf		
— <sup>175</sup> Hf			— <sup>252</sup> Cf		
— <sup>181</sup> Hf			— <sup>254</sup> Es		
— <sup>182</sup> Ta					
— <sup>185</sup> W					
— <sup>187</sup> Re					
— <sup>195</sup> Au					
— <sup>203</sup> Hg					
— <sup>204</sup> Tl					

**PART V  
LABELING**

		Yes	no
1. Are containers marked with the waste generation date?			
2. Does container have CERCLA label?			
3. Does container have WTS label?			
4. PCB Containing Waste (40 CFR 761.45)?			
Large PCB Mark (M <sub>L</sub> ) [for large containers]	Small PCB Mark (M <sub>S</sub> ) [used for small containers]		

Waste Type	PART VI PACKAGING TYPE			
	55 Gallon Drums Or other sized steel drums	Roll Off Containers <sup>a</sup>	Crosslink Polyethylene Tanks (storage) Or tanker truck (transport) VCT <sup>b</sup> VOT <sup>c</sup>	INEEL Wood Boxes <sup>a</sup> 2 x 4 x 8 ft 4 x 4 x 4 ft 4 x 4 x 8 ft
Hazardous	XX	XX	—	XX
RAD <sup>d</sup>	XX	XX	—	XX
RAD & Mixed RAD <sup>d</sup>	XX	XX	—	XX
Asbestos-TSCA	XX	XX	—	XX
Asbestos-TSCA/RAD Waste	XX	XX	—	XX
Purge Water	—	—	XX	XX
Case-by-Case <sup>d</sup>	XX	XX	XX	XX

- a. Drums, roll-offs, and INEEL wood boxes will be lined with polyethylene liners (or supersacks).
- b. Low-level radioactive waste shall be packaged for disposal in accordance with 10 CFR 61.56(a). The container must also be surveyed to ensure occupational exposures to radiation are < 500 mR/h at 1 meter for the exterior of the container. If the container's radiation level is > 500 mR/h then the container must be shielded by other containers within the SSA.
- c. VCT (Vertical Closed Top) and VOT (Vertical Open Top) above ground tanks will meet or exceed ASTM D 1998-91, Type I: Tanks molded from crosslinkable polyethylene.
- d. Wastes accepted on a case-by-case basis could require special container requirements. Therefore, the generator must verify proper containers with 49 CFR 101, Subpart C

NOTE: Other types of containers may be used if they have received approval prior to shipment.



# **Appendix B**

## **Fissionable Material Content Limits**



## APPENDIX B

### FISSIONABLE MATERIAL CONTENT LIMITS

The following describes the limits for fissionable material content in waste packages sent to the ICDF covered by criteria provided in Chapters 3.0 through 6.0.

#### B-1. EXEMPT MATERIALS

The following materials are exempt from criticality safety controls at all TSD units (HNF-PRO-537):

- 15 grams or less of any combination of  $^{233}\text{U}$ ,  $^{235}\text{U}$ ,  $^{237}\text{Np}$ ,  $^{238}\text{Pu}$ ,  $^{239}\text{Pu}$ ,  $^{240}\text{Pu}$ ,  $^{241}\text{Pu}$ ,  $^{242}\text{Pu}$ , and  $^{241}\text{Am}$
- 2 grams or less of any fissionable nuclide with atomic number greater than or equal to 95 (excluding  $^{241}\text{Am}$ )
- Depleted or natural uranium in any amount (i.e., uranium containing less than 0.72 weight percent  $^{235}\text{U}$ ).

#### B-2. NONEXEMPT MATERIALS IN STANDARD CONTAINERS APPLICABLE TO ICDF COMPLEX

Certain nonexempt materials in standard packaging configurations are acceptable at the ICDF landfill.

The fissionable material limits shall be expressed in  $^{239}\text{Pu}$  FGEs. Table B-1 is used to determine the total quantity of fissionable material in a waste container by multiplying the gram quantity of each listed isotope by the correction factor and summing the results.

Specific container limits are shown in Table B-2. Note that some of the limits in Table B-2 are based on criticality prevention requirements. Higher quantities of fissionable nuclides could be allowed on a case-by-case basis.

#### B-3. NONEXEMPT QUANTITIES OF URANIUM-BEARING WASTE APPLICABLE TO CWC, LLBG, AND WRAP

This section only applies to uranium-bearing waste where the uranium is enriched to 0.72%  $^{235}\text{U}$  or greater and the total quantity of fissionable material per container exceeds that listed in Table B-2. Use of these limits for uranium-bearing waste exceeding 1% enrichment requires that the uranium be in an insoluble or stabilized form.

For uranium-bearing waste that contains uranium in a single enrichment, the limits of Table B-3 shall apply to each container in a shipment. For criticality control, other transportation limits might apply to the entire shipment.

For uranium-bearing waste containers that have uranium in a variety of different enrichments or contain any other isotopes listed in Table B-1, the fissionable material allowed shall be determined by the sum-of-fractions method as follows:

- The total U quantity present (grams) for each enrichment will be divided by the total quantity allowed (second column in Table B-3). Enrichments shall be conservatively rounded up to the next higher value listed. The result is the uranium limit fraction (ULF).
- A uranium limit fraction shall be determined for each enrichment.
- All the uranium limit fractions are summed, the total must be less than or equal to 1.

The limit for all fissionable isotopes, other than  $^{235}\text{U}$  in the waste matrix, shall be determined if the total FGE (excluding  $^{235}\text{U}$ ) greater than 1 for these isotopes (it is neglected if the total FGE is less than or equal to 1). This nonuranium limit fraction (NLF) shall be determined as follows:

$$NLF = \frac{FGE(\text{without } ^{235}\text{U})}{100 FGE}$$

The nonuranium limit fraction must be less than or equal to 1.

The container limit fraction is determined by adding the total uranium limit fraction and the nonuranium limit fraction. The container limit fraction must be less than or equal to 1.

#### B-4. NONEXEMPT QUANTITIES OF FISSIONABLE RADIONUCLIDES IN OTHER CONFIGURATIONS

Limits for other configurations than those shown in Sections 2 and 3 may be requested as described in the text, Section 5.4.3.

Table B-1. Plutonium-239 fissile gram equivalent correction factors.

Isotope	Correction Factor	Isotope	Correction Factor
$^{233}\text{U}^a$	1.0 E+00	$^{242}\text{Am}$	3.46 E+01
$^{235}\text{U}^b$	1.0 E+00	$^{243}\text{Am}$	1.29 E-02
$^{237}\text{Np}^c$	1.5 E-02	$^{243}\text{Cm}$	5.0 E+00
$^{238}\text{Pu}^c$	1.13 E-01	$^{244}\text{Cm}$	9.00 E-02
$^{239}\text{Pu}^c$	1.0 E+00	$^{245}\text{Cm}$	1.50 E+01
$^{240}\text{Pu}^c$	2.25 E-02	$^{247}\text{Cm}$	5.00 E-01
$^{241}\text{Pu}^c$	2.25 E+00	$^{249}\text{Cf}$	4.50 E+01
$^{242}\text{Pu}^c$	7.50 E-03	$^{251}\text{Cf}$	9.00 E+01
$^{241}\text{Am}$	1.87 E-02		

- $^{233}\text{U}$  is normally negligible unless the materials have been enriched in  $^{233}\text{U}$ .
- $^{235}\text{U}$  is not included in calculating FGE unless it is enriched (greater than or equal to 0.72 wt%  $^{235}\text{U}$  in uranium).
- For conservatism, all plutonium is normally considered to be  $^{239}\text{Pu}$  unless the isotopic composition is known.

Sources: DOE/WIPP 89-004 (1996) and ANSI/ANS 8.15.

Table B-2. Fissionable material content limits for certain standard containers.

Container Type	Fissionable Material Content <sup>a</sup>
208-L (55-gal) or larger steel drum, where fissile material is contained in 20% or more of the container volume	177 FGE <sup>b</sup>
208-L (55-gal) or larger steel drum, where fissile material is contained in less than 20% of the container volume	100 FGE <sup>b</sup>
208-L (55-gal) lead-lined steel drum	100 FGE <sup>b</sup>
DOT or NRC -approved containers (e.g., DOT Specification 6M)	Maximum fissile content may not exceed that which is acceptable for transportation as specified in the DOT regulations or the NRC Certificate of Compliance
Steel box containing flushed and drained equipment and/or HEPA filters: all of the following limits shall apply:	<ul style="list-style-type: none"> <li>• 325 FGE per piece of equipment</li> <li>• 353 FGE/m<sup>3</sup> (10 EGE/ft<sup>3</sup>) on HEPA filters</li> <li>• 15 FGE in waste other than equipment or HEPA filters</li> <li>• 250 FGE total in box larger than 0.76 × 0.76 × 0.76 m (2.5 × 2.5 × 2.5 ft)</li> <li>• 325 FGE total in box larger than 0.9 × 1.4 × 1.5 m (3 × 4 × 5 ft)</li> <li>• 1,000 FGE total in box larger than 1.2 × 1.2 × 2.1 m (4 × 4 × 7 ft)</li> </ul>
<p>a. Some of these limits are based on WRAP criticality prevention requirements, which are more restrictive than CWC and LLBG limits. Higher quantities of fissionable nuclides could be allowed on a case-by-case basis for containers that will not require reprocessing at WRAP.</p> <p>b. This limit assumes that the steel drum weighs a minimum of 23 kg (50.7 lb) excluding the liner. Any drum that weighs less than 23 kg (50.7 lb) requires overpacking or completion of a criticality safety evaluation.</p>	
Source: CPS-D-149-00001, CPS-SW-149-00002, CPS-SW-149-00003, WRPI-CPS-001.	

Table B-3. Maximum uranium content of containers with uranium-bearing waste.

Maximum Enrichment (weight percent $^{235}\text{U}$ ) <sup>a</sup>	Maximum Total Uranium (kilogram)
0.71	Unlimited
1.00	450
1.15	200
1.25	141
1.50	84
1.70	61
1.80	52
2.00	40
2.50	25
3.00	18
3.50	14
1.0	11
5.0	7.8
8.0	4.0
10.0	2.9
20.0	1.2
30.0	0.75
50.0	0.41
75.0	0.25
Greater than 75.0	0.18

- a. Uranium-bearing waste disposed at trenches 31 and 34 in the LLBG cannot exceed 1% enrichment unless it is shown to be in an insoluble or stabilized form. A case-by-case evaluation will be performed by WMH acceptance organization for non-exempt uranium bearing waste exceeding 1% enrichment for trenches 31 and 34.

Sources: CPS-D-149-00001, CPS-SW-149-00002, CPS-SW-149-00003, WRPI-CPS-001.

## B-5. REFERENCES

ANS 8.15, "Nuclear Criticality Control of Special Actinide Elements," American Nuclear Society, 1981.